3. How often the collection is required: Required reports are collected and evaluated on a continuing basis as events occur. There is a one-time submittal of information to receive a license. Renewal applications are submitted every 5 years. Information submitted in previous applications may be referenced without being resubmitted. In addition, recordkeeping must be performed on an on-going basis.

4. Who is required or asked to report: All persons applying for or holding a license to manufacture, produce, transfer, receive, acquire, own, possess, or use radioactive byproduct material.

5. The number of annual respondents: 6,089 NRC licensees and 12,178 Agreement State licensees.

6. The number of hours needed annually to complete the requirement or request: Approximately 8 hours annually per licensee or 48,837 hours for the NRC licensees and 98,256 hours for the Agreement State licensees.

7. Abstract: 10 CFR Part 30 establishes requirements that are applicable to all persons in the United States governing domestic licensing of radioactive byproduct material. The application, reporting and recordkeeping requirements are necessary to permit the NRC to make a determination whether the possession, use, and transfer of byproduct material is in conformance with the Commission's regulations for protection of the public health and safety.

Submit, by April 22, 1996, comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?

2. Is the burden estimate accurate?
3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW, (Lower Level), Washington, DC. Members of the public who are in the Washington, DC, area can access this document via modem on the Public Document Room Bulletin Board (NRC's Advance Copy Document Library), NRC subsystem at FedWorld, 703–321–3339. Members of the public who are located outside of the Washington, DC, area can dial FedWorld, 1-800-303-9672, or use the FedWorld Internet address: fedworld.gov (Telnet). The document

will be available on the bulletin board for 30 days after the signature date of this notice. If assistance is needed in accessing the document, please contact the FedWorld help desk at 703–487–4608.

Comments and questions may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T–6 F33, Washington, DC, 20555–0001, or by telephone at (301) 415–7233, or by Internet electronic mail at BJS1@NRC.GOV.

Dated at Rockville, Maryland, this 9th day of February, 1996.

For the Nuclear Regulatory Commission. Gerald F. Cranford,

Designated Senior Official for Information Resources Management.

[FR Doc. 96–3694 Filed 2–16–96; 8:45 am] BILLING CODE 7590–01–P

## Reconsideration of Nuclear Power Plant Security Requirements Associated With an Internal Threat; Issued

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of issuance.

**SUMMARY:** The Nuclear Regulatory Commission (NRC) has issued Generic Letter 96-02 to notify licensees of nuclear power plants that the NRC has reconsidered its positions on certain security measures associated with protecting nuclear power plants against an internal threat. Licensees may take actions, as appropriate, after reviewing the information contained in the generic letter for applicability to their facilities. However, staff suggestions regarding potential changes to security plans are not NRC requirements; therefore, no specific action or written response is required. This generic letter is available in the Public Document Rooms under accession number 9601230206.

**DATES:** The generic letter was issued on February 13, 1996.

ADDRESSES: Not applicable.

FOR FURTHER INFORMATION CONTACT: Loren L. Bush at (301) 415–2944 or Robert F. Skelton at (301) 415–3208.

SUPPLEMENTARY INFORMATION: None.

Dated at Rockville, Maryland, this 13th day of February, 1996.

For the Nuclear Regulatory Commission. Theodore R. Quay,

Acting Director, Division of Reactor Program Management, Office of Nuclear Reactor Regulation.

[FR Doc. 96–3690 Filed 2–16–96; 8:45 am] BILLING CODE 7590–01–P

## Proposed Generic Letter: Periodic Verification of Design-Basis Capability of Safety-Related Motor-Operated Valves (M93706); Opportunity for Public Comment

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of opportunity for public comment.

**SUMMARY:** The Nuclear Regulatory Commission (NRC) is proposing to issue a generic letter to (1) more explicitly address the need for the periodic verification of the capability of safetyrelated motor-operated valves (MOVs) to perform their safety functions consistent with the current licensing bases of nuclear power plants, (2) request that each licensee establish a program, or ensure the effectiveness of a current program, at each facility within its purview, to verify on a periodic basis that safety-related MOVs continue to be capable of performing their safety functions within the current licensing bases of the facility, and (3) require that licensees provide written responses to the generic letter relating to implementation of the requested actions.

NRC regulations require that components important to the safe operation of a nuclear power plant, including MOVs, be treated in a manner that provides assurance of their performance. Appendix A, "General Design Criteria for Nuclear Power Plants," and Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," to Part 50 of Title 10 of the Code of Federal Regulations (10 CFR Part 50) include broad-based requirements in this regard. In 10 CFR 50.55a(f), the NRC requires licensees to comply with Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code).

Nuclear power plant operating experience, valve performance problems and MOV research have revealed that the focus of the ASME Code on stroke time and leak-rate testing for MOVs was not sufficient in light of the design of the valves and the conditions under which they must function. For this reason, on June 28, 1989, the NRC staff issued Generic Letter (GL) 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance." In GL 89–10, the staff requested that licensees and permit holders ensure the capability of MOVs in safety-related systems to perform their intended functions by reviewing MOV design bases, verifying MOV switch settings initially and periodically, testing MOVs under